



Journal of applied research and technology

ISSN: 1665-6423

UNAM, Centro de Ciencias Aplicadas y Desarrollo Tecnológico

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Journal of applied research and technology, vol. 17, no. 3, 2019, May-June, pp. 155-162
UNAM, Centro de Ciencias Aplicadas y Desarrollo Tecnológico

DOI: <https://doi.org/10.22201/icat.16656423.2019.17.3.808>

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A new methodology to optimize operation cycles in a BWR using heuristic techniques

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Abstract: A new system to optimize fuel assembly design, fuel reload design and control rod patterns design is shown. Fuel assembly optimization is made in two steps. In the first one, a recurrent neural network for the fuel lattice design of the bottom of the fuel assembly is used. In the second one, the top of the fuel assembly is built adding gadolinia to bottom fuel lattice. Fuel reload is optimized by another recurrent neural network whereas the control rod patterns are optimized by an ant colony method. This new system starts building a fresh fuel batch. Later, a seed fuel reload is optimized according to a Haling calculation. Afterwards an iterative process is started: firstly, control rod patterns through the cycle are optimized, once that a new fuel reload with previously optimized control rod patterns is found. If thermal limits cannot be satisfied in this iterative process after several iterations, a new seed fuel reload is designed. If cold shutdown margin cannot be fulfilled, then gadolinia concentration is increased into the fuel assembly. Finally, if energy requirements cannot be fulfilled, then the uranium enrichment of the fuel lattice of the bottom fuel assembly is increased. Results of this new system are successful: thermal limits and cold shutdown margin are fulfilled, and energy requirements are reached.

Keywords: BWR, heuristic techniques, optimization

1. INTRODUCTION

In-core fuel management tasks for BWR are fuel lattice optimization, fuel bundle design, fuel reload optimization and control rod patterns optimization. These combinatorial optimization problems are difficult to solve due to two aspects: they have large search spaces and reactor simulation codes spend too much computation

computation time. A methodology to solve these problems must find an acceptable solution in a reasonable time.

In order to design an operation cycle, the nuclear fuel lattice is the first problem to be optimized. The second one is to design the fuel assembly. Fuel reload and control rod patterns are two problems that should be solved in a coupled way. In the fuel reload design, both fresh and spent fuel assemblies are distributed into the reactor core in order to have a constant and uniform power distribution. That is verified by the thermal limits, the keff value and the fulfillment of energy requirements. However, the fulfillment

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Peer Review under the responsibility of Universidad Nacional Autónoma de México.

However, the fulfillment of the generated energy is only known until control rod patterns are designed. Control rod patterns cannot be designed if fuel reload is not known. Therefore, both problems must be optimized in a coupled way. Traditionally, both problems have been solved in a separated way. Firstly, fuel reload is optimized using a Haling calculation to give a rough estimation of the end of the cycle. Then control rod patterns are optimized.

Some papers about these problems can be found in the literature, for example, genetic algorithms (François & López, 1999; Sobolev, Gazetdinov, & Samokhin, 2017), neural networks (Ortiz & Requena, 2004), firefly algorithms (Poursalehi, Zolfaghari, & Minuchehr, 2015), tabu search (Hill & Parks, 2015) and (Castillo et al., 2004), ant colony optimization (Esquivel-Estrada, Ortiz-Servin, Castillo, & Perusquía, 2011), swarm intelligence (Ahmad & Ahmad, 2018) have been used to solve fuel reload optimization making a rough estimation of the end of the cycle. On the other hand, ant colony optimization (Ortiz & Requena, 2006), neural networks (Mejía & Ortiz, 2005), genetic algorithms (Montes, Ortiz, Requena, & Perusquía, 2004) and tabu search (Castillo, Ortiz, Alonso, Morales, & Del Valle, 2005) have been used to optimize control rod patterns for a given fuel reload. There are few works where both problems have been solved in a coupled way (Kobayashi & Aiyoshi, 2002; Ottinger & Maldonado, 2015).

With respect to fuel lattice optimization, several heuristic techniques have been used: tabu search (François, Martín-del-Campo, François, & Morales, 2003), ant colony optimization (Montes, François, Ortiz, Martín-del-Campo, & Perusquía, 2011), path relinking (Castillo, Ortiz-Servin, Perusquía, & Silvestre, 2011), genetic algorithms (Martín-del-Campo, François, Carmona, & Oropeza, 2007) and neural networks (Ortiz, Castillo, Montes, Perusquía, & Hernández, 2009), among others. Fuel assembly design has been solved using Montecarlo method (Tohjoh, Watanabe, & Yamamoto, 2006) and the block coordinate descent method (Tung, Lee, Kuo, & Yaur, 2017). This paper is divided in the following sections: in Section 2 a brief description of the problem is provided. In the third section the optimization system is described. In Section 4 results for an equilibrium cycle are shown. Finally, conclusions, acknowledgements and references are shown.

2. PROBLEM DESCRIPTION

In Figure 1 a typical reactor core is shown. Each box represents a reactor channel and each cross represents a control rod. Four channels and the control rod in the middle of them are named a control lattice. Control rods in red are used to operate the reactor, black control rods remain withdrawal of reactor core during the cycle operation. Fuel assemblies are introduced into the reactor channels.

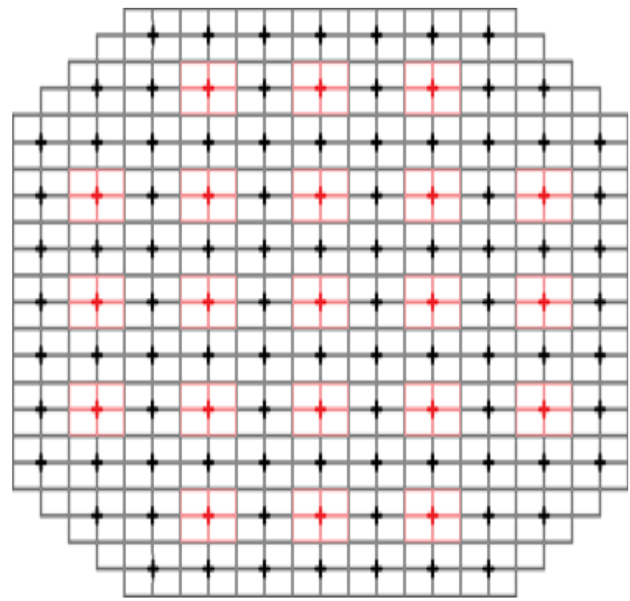


Fig. 1. A typical transverse view of a reactor core.

A fuel assembly is a square prism, which is divided in 25 axial nodes. Each node corresponds to a fuel lattice. In Figure 2 a typical fuel lattice is shown.

From Figure 2 in small boxes a pellet with uranium or a mixture of uranium and gadolinia can be introduced. Big boxes are water channels. Up to the left, a control rod is shown.

The reactor core operation time is divided in two kind of periods: the electricity generation period (named operation cycle) and the refueling period (the most spent fuel assemblies are replaced by the fresh fuel assemblies). Previously to start a cycle operation, a design stage is needed. In that stage, fresh fuel assemblies, fuel reload, and control rod patterns are optimized. In the remainder of the section, brief descriptions of these optimization problems are shown.

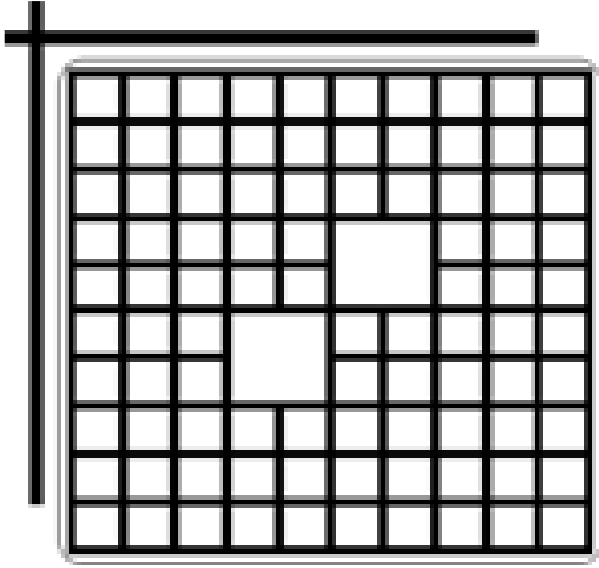


Fig. 2. Typical 10x10 fuel lattice.

2.1 FUEL LATTICE DESIGN

Enriched uranium is introduced to the reactor core in the form of pellets. Gadolinia (Gd_2O_3) is a neutron absorbent material and it is used as a burnable poison in the reactivity core control. The problem is to allocate pellets in the small boxes in Figure 2 in order to minimize the local power peaking factor ($LPPF$) subject to:

$$k_t^L < kinf < k_t^U \quad (1)$$

and

$$\frac{\sum_{i=1}^{92} u_i}{92} = U_{tar} \quad (2)$$

where $kinf$ is the infinite multiplication factor of neutrons; k_t^L and k_t^U are the minor and major $kinf$ permitted values; u_i is the uranium enrichment of each one of the 92 pellets; and U_{tar} is the required uranium enrichment of the fuel lattice. Uranium enrichment and gadolinia concentrations can be chosen of the elements of the Table 1.

2.2 FUEL ASSEMBLY DESIGN

As it was said, a fuel assembly is a vertical array of 25 positions. One node at the bottom and two nodes at the top of the fuel assembly typically contain fuel lattices with natural uranium, in order to reflect neutrons into the core.

The other 22 nodes are divided in three zones, where each zone has a different fuel lattice. A typical zones distribution is the following: bottom zone has 10 nodes; second zone has 6 nodes and the top zone has 8 nodes. Normally, the fuel lattice of the bottom zone is optimized, and the upper zones have the same uranium enrichment but different gadolinia concentration.

Table 1. Uranium enrichment and gadolinia concentrations inventory.

U	$Gd_2O_3 = 0\%$	$Gd_2O_3 = 4\%$	$Gd_2O_3 = 5\%$	$Gd_2O_3 = 6\%$
2.00%	✓			
2.40%	✓			
2.80%	✓			
3.20%	✓			
3.60%	✓	✓	✓	✓
3.95%	✓	✓	✓	✓
4.20%	✓	✓	✓	✓
4.40%	✓			
4.90%	✓			

2.3 FUEL RELOAD DESIGN

At the end of the cycle, the most spent fuel assemblies are substituted by fresh fuel assemblies. Fresh and spent fuel assemblies must be re-allocated into the core in order to maximize the energy production subject to:

$$FLPD_{current} < FLPD_{max} \quad (3)$$

$$MAPRAT_{current} < MAPRAT_{max} \quad (4)$$

$$FLCPR_{current} < FLCPR_{max} \quad (5)$$

$$SDM_{current} > SDM_{min} \quad (6)$$

where $FLPD_{current}$ and $FLPD_{max}$ are the current and the maximum fraction to linear power density, $MAPRAT_{current}$ and $MAPRAT_{max}$ are the current and the maximum fraction to the average planar linear generation rate, $FLCPR_{current}$ and $FLCPR_{max}$ are the current and the maximum fraction to the critical power ratio and $SDM_{current}$ and SDM_{min} are the current and the minimum

cold shutdown margin. $FLPD_{current}$, $MAPRAT_{current}$, $FLCPR_{current}$ and $SDM_{current}$ are calculated by SIMULATE-3 (Dean, 2005).

2.4 CONTROL ROD PATTERNS DESIGN

Control rods are used to compensate the burnup of the uranium in order to keep the chain reaction. A control rod can be positioned in 25 axial places. Normally, 6 intermediate axial positions are not used due to when a control rod is put in these positions, the axial power distribution is distorted. Thus, the problem is to determine the axial positions of the red control rods (see Figure 1) in order to minimize the differences between a target power distribution and the current power distribution subject to restriction of Equations 3, 4 and 5. In addition, reactor critically must be fulfilled.

3. OPTIMIZATION SYSTEM DESCRIPTION

In order to optimize the cycle operation, a new system was developed. A recurrent neural network to optimize fuel lattices was used. A local search technique to build the fuel assembly was used. Another recurrent neural network to optimize the fuel reload was used. Finally, an ant colony to optimize the control rod patterns was used. In the rest of the section, the description of the way the heuristic techniques were used to solve these problems is shown.

3.1 FUEL LATTICE OPTIMIZATION WITH A RECURRENT NEURAL NETWORK

The Hopfield's neural network is a recurrent neural network. The response of the neurons is a bi-valued output. This kind of neural network has associated an energy function that measures the energy of the neural network. With appropriate changes in the responses of the neurons, the energy can be minimized. A generalization of this model was proposed by Mérida-Casermeiro, Galán-Marín, and Muñoz-Perez (2011). The responses of neurons can be multi-values in the interval $[0, \dots, N]$, where N is the number of elements in the Table 1. For fuel lattice optimization, the recurrent neural network has 47 neurons equivalent to 47 pellets that can be introduced into a half fuel lattice. A half fuel lattice can

be seen when a line from control rod corner to corner without control rod in Figure 2 is drawn. Besides, a water channel is formed by the space of 4 positions. The response of neurons corresponds with an uranium enrichment and gadolinia concentrations of Table 1. The set of neurons' responses is named the global neural state (GNS). The neurons have to change their responses in order to minimize the energy of the neural network. The energy function to solve the fuel lattice optimization is the following:

$$FE = w_1 LPPF(GNS) + w_2 |k_{inf}^{tar} - kinf(GNS)| \quad (7)$$

where k_{inf}^{tar} is the target $kinf$ value, and is determined previously with the linear reactivity model (Driscoll, Downar, & Pilat, 1990); $LPPF(GNS)$ and $kinf(GNS)$ means that these lattice parameters depends on global neural state. These variables are calculated by CASMO-4 code (Edenius, 2004). For more details about this neural network and its implementation to solve this problem, you can read Ortiz et al. (2009).

3.2 FUEL ASSEMBLY BUILDING WITH A LOCAL SEARCH TECHNIQUE

Upper zones in the fuel assembly are built using the pellets distribution found by the recurrent neural network in the previous step. Only gadolinia concentrations are changed. The space solutions to change gadolinia in the just optimized fuel lattice is small. So, a sophisticated optimization heuristic to solve this problem is not required. It is enough to implement a simple local search and to choose the one that has the minimum LPPF value.

3.3 FUEL RELOAD OPTIMIZATION WITH A RECURRENT NEURAL NETWORK

Another recurrent neural network with 111 neurons to optimize the fuel reload was used. In a quarter reactor core (see the Figure 1), there are 111 channels. Fresh and burnt fuel assemblies are sort according to their burnup level and an integer number is assigned to each one. The response of neurons is an integer number in the interval of $[1, \dots, M]$, where M is the number of the channels in the

quarter core. So, if neuron A has the integer B, like response, it means that in channel number A the fuel assembly number B (according to burnup level) is introduced. Also, the global neural state is named to the set of neuron's response. A global neural state is a possible fuel reload to be used in the reactor core. Responses of neurons have to be changed in order to minimize the energy value of the recurrent neural network. That energy function is the following:

$$FE = C - w_1 keff(GNS) - w_2(FLPD_{max} - FLPD(GNS)_{Curr}) - w_3(MAPRAT_{max} - MAPRAT(GNS)_{Curr}) - w_4(FLCPR_{max} - FLCPR(GNS)_{Curr}) - w_5(SDM_{Curr} - SDM(GNS)_{Min}) \quad (8)$$

where C is a positive constant in order to do not have negative values in energy function; keff is the effective multiplication factor of neutrons at the end of the cycle; $FLPD(GNS)_{Curr}$, $MAPRAT(GNS)_{Curr}$, $FLCPR(GNS)_{Curr}$ and $SDM(GNS)_{Curr}$ depend on global neural state and were previously defined; $FLPD_{max}$, $MAPRAT_{max}$, $FLCPR_{max}$ and SDM_{Min} are the limit values to the thermal limits and the cold shutdown margin, respectively; w_i are weighting factors.

3.4 CONTROL RODS PATTERNS OPTIMIZATION WITH AN ANT COLONY SYSTEM

Ant colony system emulates the behaviour of real ants and their ability to build short paths between their nest and the food sources. Real ants can build such short path because they communicate between them depositing a substance named pheromone (ρ). When a real ant finds a pheromone trail, the probability to follow it is high. The ants spend more time to go through long trails and the pheromone deposit rate is lower than short trails. On the other hand, the pheromone evaporates with the passage of the time. So, long trails disappear easily, and short trails endure. To optimize control rods patterns, red control rods in a quarter core will be used (see Figure 1), it is to say 8 control rods. The artificial ants must decide an axial

position to insert each one of 8 control rods in the core. The artificial ants deposit pheromone in each axial position depending on the convenience of the axial position. The most convenient axial positions have more pheromone than those with less convenience to be used. For example, if an axial position helps to mitigate a peak power or kept the reactor in critical state, it is more convenient to be chosen. For each axial position is possible to define a convenience level (Λ) to mitigate power peaks or the control rod worth. An artificial ant decides to use an axial position in function of the pheromone deposits and the convenience level. Ant colony system uses three rules to operate and they are showing up next:

$$S = S_i^j | \max(\rho(i, j) \Lambda(i, j)^\beta) \quad (9)$$

the i-th control rod will be inserted in the j-th axial position with maximum product of pheromone and convenience level. β is a constant greater than zero. The pheromone is updated by each ant once it has chosen the j-th axial position for i-th control rod. It is done with the following equation:

$$\rho(i, j) = (1 - \varepsilon)\rho(i, j) + \varepsilon\rho_0 \quad (10)$$

where ε is the evaporation rate whose values are in the interval $([0, 1])$, the pheromone is updated by fixing increments denoted by ρ_0 . Once that all ants have proposed a control rod pattern, they are evaluated by SIMULATE-3 code (Dean, 2005). The best control rod pattern receives an extra pheromone, and it is updated according to:

$$\rho(i, j) = (1 - \varepsilon)\rho(i, j) + \frac{E}{F} \quad (11)$$

$$F = w_1 | keff_{Curr} - keff_{tar} | - w_2(FLPD_{max} - FLPD_{Curr}) - w_3(MAPRAT_{ma} - MAPRAT_{Curr}) - w_4(FLCPR_{max} - FLCPR_{Curr}) \quad (12)$$

where $keff^{ar}$ is the target $keff$ and w^i are weighting factors.

3.5 FLOW DIAGRAM

In Figure 3 the flowchart of the system is shown.

System starts optimizing the fuel lattice of the bottom fuel assembly according to the average uranium enrichment and gadolinia requirements. Then upper fuel lattices are found with the local search technique. An initial fuel reload is obtained with the recurrent neural network. The involved variables in Eq. 8 are obtained with SIMULATE-3 running a Haling calculation at the first iteration of the system. The next step is to optimize control rods patterns with the ant colony system. If thermal limits are not fulfilled, then a new fuel reload is optimized. Control rod patterns are simulated by SIMULATE-3 to obtain the variables of Eq. 8 from second iteration until a stop criterion is fulfilled. If cold shutdown margin is not fulfilled, then gadolinia concentration is increased and a new fuel

assembly is made. Finally, if energy requirements are not reached, then the average uranium enrichment is increased 0.01% U235 and a new fuel lattice must be optimized.

4. RESULTS

In order to test the capabilities of the new system, a cycle operation of 18 months of cycle length was used. This cycle has two types of fresh fuel assemblies, both have 3.66% of uranium enrichment but different gadolinia concentration. 60 fresh fuel assemblies of the type one are loaded into the core and this type is named FA1. 52 fuel assemblies of the type two are loaded into the core and the type is named FA2. This cycle has a length equal to 10.9 GWD/MT. For that burnup, the target keff is equal to 0.9972. $FLPD_{max}$, $MAPRAT_{max}$ and $FLCPR_{max}$ are equal to 0.96. SDM_{min} is equal to 1.2%.

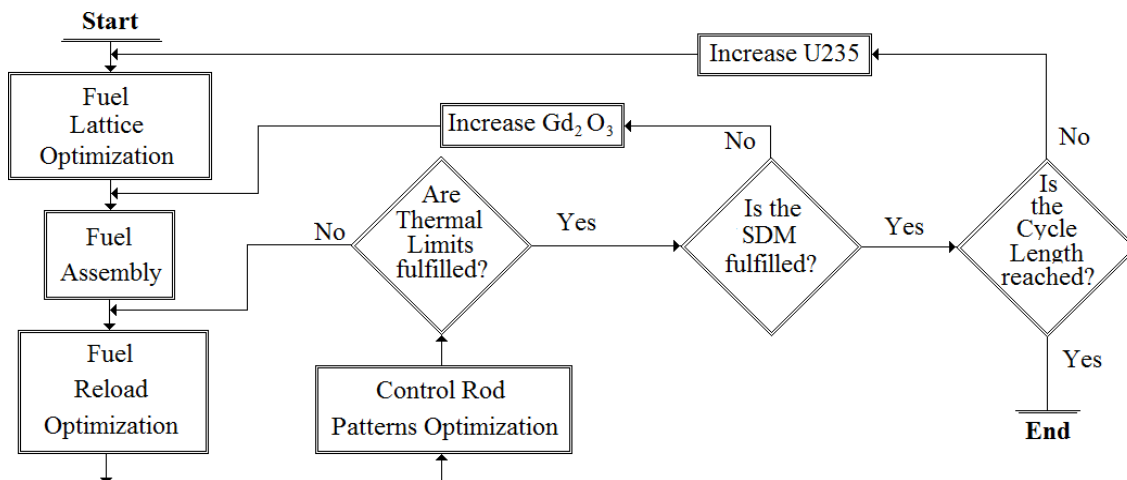


Fig. 3. General flowchart.

Table 2. Core parameters of cycle operation.

Case	LPPF	kinf	FLPD	MAPRAT	FLCPR	SDM%	keff
1	1.215	1.17012	0.951	0.902	0.908	1.210	0.99720
2	1.246	1.16419	0.949	0.938	0.889	1.513	0.99770
3	1.275	1.17519	0.942	0.867	0.932	1.515	0.99753
4	1.317	1.08461	0.958	0.948	0.936	1.536	0.99785
5	1.255	1.16759	0.945	0.922	0.897	1.720	0.99927
6	1.257	1.17211	0.939	0.859	0.884	1.552	0.99970
7	1.250	1.14790	0.942	0.867	0.932	1.515	0.99753
8	1.277	1.17784	0.951	0.927	0.914	1.547	0.99843
Ref			<0.960	<0.960	<0.960	>1.200	≥0.9972

In Table 2 core and fuel lattice's parameters of several executions of the system are shown. As can be seen, thermal limits of all executions are lower than the upper limit (0.960), also all cold shutdown margins values are greater than 1.2%. The required keff to guarantee the energy specifications is 0.9972. In all executions the keff values are greater than the required value. So, system can find a complete solution of all involved problems in the in-core fuel management. Besides, cases 5, 6 and 8 have a high enough keff value to generate more energy than 10.9 GWD/MT.

In Figure 4, a typical evolution of the fuel lattice optimization is shown. It can be seen that the energy function value decreases when the number of iterations increases and also LPPF value is decreased. In Figure 5, a typical evolution of the parameters of a fuel reload optimization is shown. It can be seen that the energy function value decreases when the number of iterations increases, and also thermal limits are decreased below 0.96.

In Figure 6, a typical pheromone evolution is shown. It can be seen that at the starting of optimization, the pheromone is accumulated in the axial position 32. However, the ants change the position of that control rod and it is inserted at axial position 02. Similar graphs are built for all control rods in the core. At the end of the optimization process, the ants assign the axial position according to the pheromone accumulation. It is difficult to graph pheromone and thermal limits evolution in the same figure, but according to pheromone is accumulated, the thermal limits and the critically state are fulfilled.

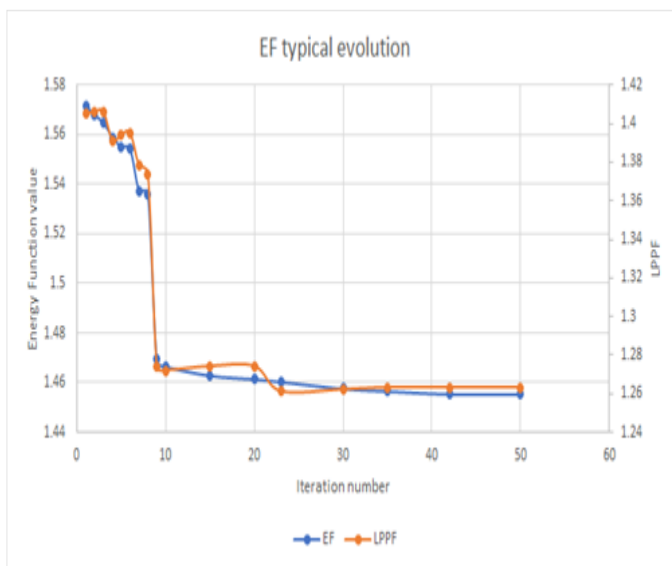


Fig. 4. Typical fuel lattice optimization.

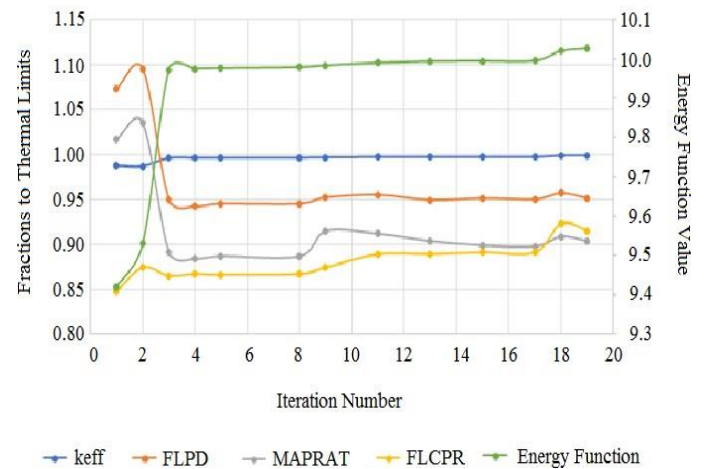


Fig. 5. Typical fuel reload optimization.

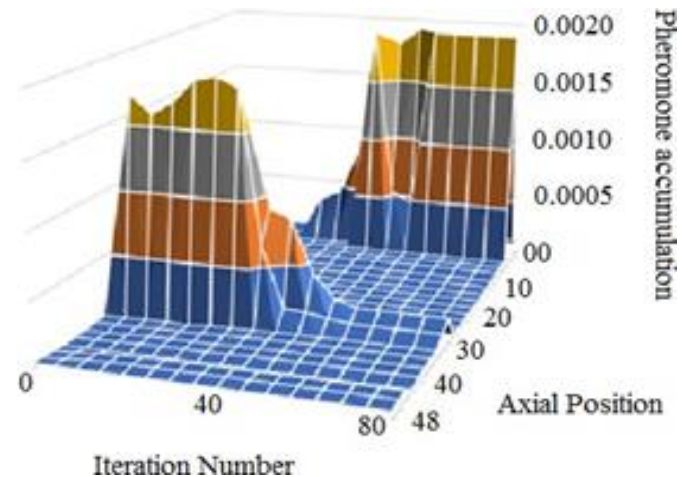


Fig. 6. Typical pheromone accumulation.

5. CONCLUSIONS

In this paper a new methodology to optimize cycle operations in BWRs was shown. The heuristic techniques called recurrent neural networks and an ant colony system were used to solve the involved combinatorial optimization problems. It was shown that the energy function of recurrent neural networks has good convergence towards minimum values. Also, pheromone accumulation works to improve thermal limits and keff value throughout the cycle. This methodology is able to find solutions to all problems of the in-core fuel management so that thermal limits, cold shutdown margin are fulfilled and energy requirements are reached.

ACKNOWLEDGEMENTS

The authors acknowledge the support given by Instituto Nacional de Investigaciones Nucleares through the research project CA-708 and Consejo Nacional de Ciencia y Tecnología through the research project CB-2011-01- 168722.

REFERENCES

- Ahmad, A. & Ahmad, S. U. I. (2018). Optimization of fuel loading pattern for a material test reactor using swarm intelligence. *Progress in Nuclear Energy*, 103, 45-50.
- Sobolev, A. V., Gazetdinov, A. S., & Samokhin, D. S. (2017). Genetic algorithms for nuclear reactor fuel load and reload optimization problems. *Nuclear Energy and Technology*, 3(3), 231-235.
- Castillo, A., Alonso, G., Morales, L. B., del Campo, C. M., Francois, J. L., & del Valle, E. (2004). BWR fuel reloads design using a Tabu search technique. *Annals of Nuclear Energy*, 31(2), 151-161.
- Castillo, J. A., Ortiz, J. J., Alonso, G., Morales, L. B., & Del Valle, E. (2005). BWR control rod design using tabu search. *Annals of Nuclear Energy*, 32(7), 741-754.
- Castillo, A., Ortiz-Servin, J. J., Perusquía, R., & Silvestre, Y. C. (2011). Fuel lattice design with Path Relinking in BWR's. *Progress in Nuclear Energy*, 53(4), 368-374.
- Dean, D. W., 2005. SIMULATE-3. Advanced Three-Dimensional Two-Group Reactor Analysis Code. User's Manual, 3rd Edition. Studsvik Scandpower INC. SSP-95/15.
- Martin-del-Campo, C., François, J. L., Carmona, R., & Oropeza, I. P. (2007). Optimization of BWR fuel lattice enrichment and gadolinia distribution using genetic algorithms and knowledge. *Annals of Nuclear Energy*, 34(4), 248-253.
- Edenius, J. R. M., 2004. CASMO-4. A Fuel Assembly Burnup Program. User's Manual, 4th Edition. Studsvik Scandpower INC. SSP-95/15.
- Esquivel-Estrada, J., Ortiz-Servin, J. J., Castillo, J. A., & Perusquía, R. (2011). Azcaxalli: A system based on Ant Colony Optimization algorithms, applied to fuel reloads design in a Boiling Water Reactor. *Annals of Nuclear Energy*, 38(1), 103-111.
- Francois, J. L., Martín-del-Campo, C., François, R., & Morales, L. B. (2003). A practical optimization procedure for radial BWR fuel lattice design using tabu search with a multiobjective function. *Annals of Nuclear Energy*, 30(12), 1213-1229.
- Francois, J. L., & Lopez, H. A. (1999). SOPRAG: a system for boiling water reactors reload pattern optimization using genetic algorithms. *Annals of Nuclear Energy*, 26(12), 1053-1063.
- Ottinger, K. E., & Maldonado, G. I. (2015). BWROPT: A multi-cycle BWR fuel cycle optimization code. *Nuclear Engineering and Design*, 291, 236-243.
- Kobayashi, Y., & Aiyoshi, E. (2002). Optimization of boiling water reactor loading pattern using two-stage genetic algorithm. *Nuclear Science and Engineering*, 142(2), 119-139.
- Driscoll, M. J., Downar, T. J. & Pilat, E. E. (1991). The Linear Reactivity Model for Nuclear Fuel Management. Illinois, USA: American Nuclear Society.
- Mejía, D., & Ortiz, J. (2005). Control rod patterns optimization using neural networks. In The 9th World Multi-Conference on Systemics, Cybernetics and Informatics (pp. 16-20).
- Montes, J. L., Francois, J. L., Ortiz, J. J., Martin-del-Campo, C., & Perusquia, R. (2011). Fuel lattice design in a boiling water reactor using an ant-colony-based system. *Annals of Nuclear Energy*, 38(6), 1327-1338.
- Montes, J. L., Ortiz, J. J., Requena, I., & Perusquía, R. (2004). Searching for full power control rod patterns in a boiling water reactor using genetic algorithms. *Annals of nuclear energy*, 31(16), 1939-1954.
- Mérida-Casermeyro, E., Galán-Marín, G., & Muñoz-Perez, J. (2001). An efficient multivalued Hopfield network for the traveling salesman problem. *Neural Processing Letters*, 14(3), 203-216.
- Hill, N. J., & Parks, G. T. (2015). Pressurized water reactor in-core nuclear fuel management by tabu search. *Annals of Nuclear Energy*, 75, 64-71.
- Poursalehi, N., Zolfaghari, A., & Minuchehr, A. (2015). A novel optimization method, Effective Discrete Firefly Algorithm, for fuel reload design of nuclear reactors. *Annals of Nuclear Energy*, 81, 263-275.
- Ortiz, J. J., Castillo, A., Montes, J. L., Perusquía, R., & Hernández, J. L. (2009). Nuclear fuel lattice optimization using neural networks and a fuzzy logic system. *Nuclear Science and Engineering*, 162(2), 148-157.
- Ortiz, J. J., & Requena, I. (2004). Using a multi-state recurrent neural network to optimize loading patterns in BWRs. *Annals of Nuclear Energy*, 31(7), 789-803.
- Ortiz, J. J., & Requena, I. (2006). Azcatl-CRP: An ant colony-based system for searching full power control rod patterns in BWRs. *Annals of Nuclear Energy*, 33(1), 30-36.
- Tohjoh, M., Watanabe, M., & Yamamoto, A. (2006). Three-dimensional pin power reconstruction for the axially heterogeneous region in BWR. *Annals of Nuclear Energy*, 33(3), 242-251.
- Tung, W. H., Lee, T. T., Kuo, W. S., & Yaur, S. J. (2017). Optimization of axial enrichment distribution for BWR fuels using scoping libraries and block coordinate descent method. *Nuclear Engineering and Design*, 313, 84-95.