

Brazilian Journal of Radiation Sciences ISSN: 2319-0612 Sociedade Brasileira de Proteção Radiológica

Alvarenga, T. A.; Fernandes, S. S.; Pereira, W. W.; Mayhugh, M. R.; Caldas, L. V. E. Estimation of scattered radiation influence on neutron beams at a calibration laboratory using Monte Carlo simulation of a long counter Brazilian Journal of Radiation Sciences, vol. 11, no. 2, e2249, 2023, April-June Sociedade Brasileira de Proteção Radiológica

DOI: https://doi.org/10.15392/2319-0612.2023.2249

Available in: https://www.redalyc.org/articulo.oa?id=722277919010



Complete issue

More information about this article

Journal's webpage in redalyc.org



Scientific Information System Redalyc

Network of Scientific Journals from Latin America and the Caribbean, Spain and Portugal

Project academic non-profit, developed under the open access initiative



OF RADIATION SCIENCES 11-02 (2023) | 01-12 | e2249





Estimation of scattered radiation influence on neutron beams at a calibration laboratory using Monte Carlo simulation of a long counter

Alvarenga^a, T. A.; Fernandes^b, S. S.; Pereira^b, W. W.;

Mayhugh^c, M, R.; Caldas^a, L. V. E.

^a Instituto de Pesquisas Energéticas e Nucleares, Comissão Nacional de Energia Nuclear, IPEN/CNEN, Av. Prof. Lineu Prestes, 2242, 05508-000, São Paulo, SP, Brazil

^b Instituto de Radioproteção e Dosimetria, Comissão Nacional de Energia Nuclear, IRD/CNEN, Av. Salvador Allende, 9, 22780-160, Rio de Janeiro, RJ, Brazil

^c Faceted Development, LLC, 3641 Rawnsdale Rd., Shaker Hts., OH 44122, USA tallysonalvarenga@gmail.com; simonesilvafernandes@gmail.com; walsanwagner@gmail.com; mikemayhugh@gmail.com; lcaldas@ipen.br

ABSTRACT

At the Neutron Calibration Laboratory (LCN) of IPEN/CNEN, a ²⁴¹AmBe source was used to test and calibrate neutron detectors. The neutrons emitted by the source reach the detector as intended, but they also scatter undesirably from the building's floor, ceiling, and walls, leading to indirect detection. A Long Counter (LC) detector was modeled using the MCNPX Monte Carlo code. The objective of this study was to measure the contribution of scattered radiation at the LCN / IPEN, and to determine the fluence rate, at different points in the calibration room at source-to-detector distances of 100 cm and 150 cm; subsequently, the results were compared with those of the Brazilian Laboratory of Metrology of Ionizing Radiation (LNMRI). The results show that the fluence rates of LCN / IPEN are comparable to those of this neutron laboratory for the 100 and 150 cm source-to-detector distances using a ²⁴¹AmBe source (37 GBq). The results show that the calibration of neutron detectors should be performed at distances less than 100 cm, where the contribution of scattered radiation is within the 40% limit recommended by ISO 8529-1.

Keywords: Long counter, Scattered radiation, Monte Carlo simulation.

ISSN: 2319-0612

DOI: https://doi.org/10.15392/2319-0612.2023.2249

Submitted: 2023-01-13 Accepted: 2023-06-02



1. INTRODUCTION

A large energy range and complex interactions make reliable neutron radiation measurement difficult. There is considerable effort to validate and improve methods and tools that perform the measurements in order to provide results with lower associated uncertainty [1, 2, 3, 4]. The main objective of calibrating neutron detectors is to ensure that measurements are accurate with known uncertainties, taking into account the requirements established by regulatory authorities. In practice, the main problem in the calibration of neutron detectors is scattered radiation from the walls, ceiling and floor of the laboratory [3, 5, 6, 7].

This scattered radiation varies depending on the size and material of the laboratory (walls, floor and ceiling). The neutron spectrum measured at a given point is not the same as the spectrum emitted by the neutron source, which directly influences the reading of the instrument to be calibrated and causes systematic calibration errors [8, 9, 10].

To choose a reliable method to assess scattered radiation, the calibration distance, the source type, and the laboratory dimensions have to be analyzed [3]. The determination of scattered radiation can be performed experimentally using the methods recommended by ISO 8529-2 [11], which are: the Semi-Empirical Method (SEM), the Reduced Fitting Method (RFM), the Generalized Fit Method (GFM) and the Shadow Cone Method (SCM), and through simulation using Monte Carlo Code (MCNP), which allows a more detailed assessment [11, 10].

To characterize the radiation field of the Neutron Calibration Laboratory (LCN) of the Instituto de Pesquisas Energéticas e Nucleares (IPEN), the influence of scattered radiation on the fluence at different calibration positions was evaluated. Although knowledge of the full neutron spectrum is desirable, it is not essential to perform the calibration of neutron radiation detectors using all neutron fields in which the detectors will be used, such as nuclear reactors or radioactive sources [7, 8, 12, 13].

It is important to highlight that each method presents its advantages and limitations, and the choice depends on the specific application and project requirements. In some cases, it may be necessary to use both methods to obtain a comprehensive characterization of the neutron spectrum and intensity.

To ensure improved reliability of the measurements performed using the Bonner Sphere Spectrometer (BSS), it is essential to compare them with a metrological standard that establishes a correlation between the average energy range generated by a neutron source (in this study, a radionuclide source) and the corresponding change in response of the device based on neutron energy [14].

The Long Counter is a secondary standard device important for the comparison and characterization of fluence fields. It is considered a secondary pattern detector because it has a relatively stable response, low sensitivity to gamma radiation, and high sensitivity to neutron radiation across a wide range of neutron energies (0.2 MeV to 20 MeV).

The instrument is based on the detection of slow neutrons in a central cylindrical proportional counter, usually BF³ or ³He, surrounded by a cylindrical polyethylene moderator [15]. The LC is widely used as a standard system by the main National Neutron Metrology Institutes in the world [16].

The efficiency of the LC for measuring neutron radiation was confirmed by studies carried out using Monte Carlo simulation, which provided an accurate view of the response obtained by the detector [17]. In this way, it is possible to compare the measurements performed with the BSS system with the results of the secondary standard (LC) for the same reference point.

The LC used in this study in the simulations was characterized at the Low-Scattering Neutron Laboratory, a facility belonging to the Ionizing Radiation Metrology Laboratory (LMNRI) of the Institute of Radioprotection and Dosimetry (IRD).

The aim of this study was to evaluate the scattered neutron radiation in the LCN/IPEN, obtaining neutron spectra and fluence rates and using LC simulation, at source-detector distances of 100 cm and 150 cm. Furthermore, this study compared these results with those obtained at the Brazilian Laboratory of Metrology of Ionizing Radiations (LNMRI), located at the Institute of Radioprotection and Dosimetry (IRD/CNEN) in Rio de Janeiro, Brazil.

2. MATERIALS AND METHODS

The neutron source used to carry out this work was ²⁴¹AmBe, type x3, 20.4 mm in diameter x 31 mm long, manufactured by Amersham International Ltd, which has been

calibrated at the LNMRI, and has an activity of 37 GBq (1 Ci) and a neutron emission rate of (2.46 ± 0.06) . 10^6 s⁻¹ [13].

The components used to compose the LCN were selected according to the PNNL-15870 report, which specifies the density of various materials. Concrete has a density of 2.35 g/cm³, granite has a density of 2.69 g/cm³, wooden doors have a density of 0.42 g/cm³, iron doors have a density of 7.874 g/cm³, and lead doors have a density of 11.35 g/cm³ [18].

The Shadow Cone Method (SCM) is the simplest technique used to correct the scattering of neutron radiation. A shadow cone positioned between the source and the detector eliminates direct neutrons, allowing only scattered neutrons to be detected.

This method has been widely used in the evaluation of scattering, as it is considered simple compared to other methods, allowing direct measurement. For the application of this method, in Annex E of the ISO 8529-2 standard [13], there is a recommendation for the use of a cone composed of neutron-absorbing material (borated polyethylene) and solid iron, with the rear part which is close to the source, that must be made up of 20 cm of solid iron and the front part of 30 cm of borated polyethylene. In Figure 1 a diagram of the Shadow Cone Method is presented [13].

Iron Polyethylene Detector

Figure 1: Diagram of the Shadow Cone Method.

Source: Khabaz and Hakimabad [19].

For each distance between the source and the detector center (l), the direct beam of neutrons, M_D , was determined from the difference between the number of counts measured without the interposed shadow cone (total contribution), M_T , the number of counts measured with the interposed shadow cone (scattered contribution), M_S , and F_A is air attenuation factor.

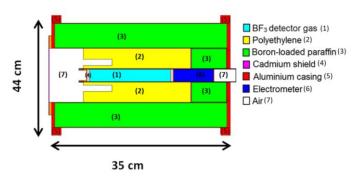
$$M_D = [M_T - M_S] \cdot F_A \tag{1}$$

The details of the composition of the materials of the Shadow-Cone in the MCNPX code were taken based from the data of the Report PNNL-15870 [18], where iron has a density of 7.8740 g/cm³ and borated polyethylene of 1.000000 g/cm³.

The MCNPX Monte Carlo code was used to perform the simulations, where the neutron spectrum used as the initial spectrum in the Monte Carlo calculations was from the ²⁴¹AmBe source described in ISO 8529-1 [20]. The LC simulation used in this work was based on the model proposed by Slaugther[15], which has a cylindrical shape of 44.0 cm in diameter and 35.0 cm in length covered with aluminum. In its internal structure there is a BF₃ detector, 26.5 cm in length with an external diameter of 4.68 cm, located in the center, covered by a cylinder of high density polyethylene, 9.9 cm thick. There is a second boron paraffin cylinder that covers these structures in order to minimize the lateral incidence of scattered neutrons.

In order to achieve highly accurate results in the LC simulation, a total of $2x10^9$ histories were simulated. This was accomplished by utilizing tally F4, which measures the fluence in a cell and is attached to the FM4 modifier card, along with code 10^7 . The purpose of this was to determine the number of reactions 10 B (n, α) 7 Li in the active volume of the detector tube. The LC geometry that was used in the simulation is shown in Figure 2.

Figure 2: Long Counter geometry used for the simulation with MCNPX. See text for dimensions.

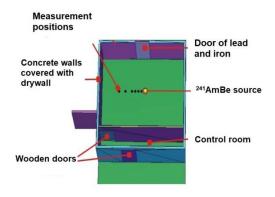


Source: Author.

The response curves for the fluence rates at the two distances were obtained using MCNPX simulations, with the neutron sources represented by monoenergetic parallel beams emitted from a circular surface with a diameter equal to the external diameter of the LC. The simulations were performed using 84 energy intervals, enabling the characterization of the device within an interval of $1x10^{-9}$.

The Neutron Calibration Laboratory (LCN) has lateral dimensions of 6.88 m x 5.46 m and walls of concrete 15 cm thick. The laboratory is 2.8 m high, the concrete ceiling is 15 cm thick, and the granite floor is 5 cm thick. The ²⁴¹AmBe source is positioned in the center of the laboratory at a height of 1.4 m. Figure 3 shows the Neutron Calibration Laboratory of IPEN and the structural laboratory geometry, where the MCNPX was applied.

Figure 3: Neutron Calibration Laboratory of IPEN and the structural laboratory geometry, where the MCNPX was applied. Dimensions: 6.88 m x 5.46 m, height of 2.8 m, concrete walls and ceiling 15 cm thick and granite floor 5 cm thick.



Source: Alvarenga et al [5].

The simulations were performed by modeling the laboratory environment with the LC positioned at the source-detector distances of 100 cm and 150 cm, with and without the shadow cone in order to evaluate the fluence rates at these locations. The neutron fluence rates obtained were compared with those from LNMRI.

3. RESULTS AND DISCUSSION

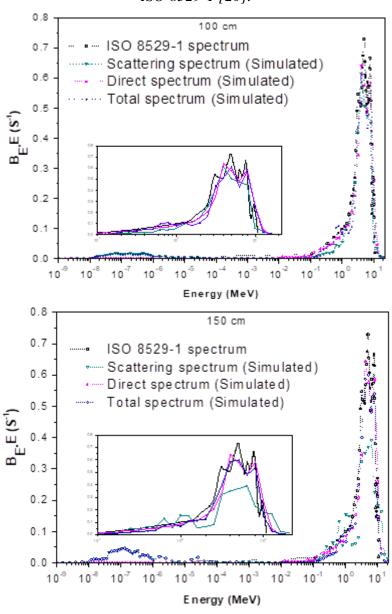
Initially the neutron spectra were obtained. Figure 4 shows the direct, scattered and total neutron fluence spectra, obtained via simulation at the source-detector distances of 100 cm and 150 cm at LCN, together with the reference spectrum of ISO 8529-1 [20], which has a maximum uncertainty in the energy bins of 3%.

The response curves for the fluence rates at the two distances are presented in Figure 4. They were determined through MCNPX simulation with the neutron sources represented

by monoenergetic parallel beams emitted from a circular surface with a diameter equal to the external diameter of the LC. The simulations were performed with 84 energy intervals.

The graphical representation is spectral source strength BE - dB/dE (fluence by lethargy) vs. energy [20].

Figure 4: Direct, scattered and total spectra, obtained by simulation at the source-detector distances of 100 cm and 150 cm, compared with the reference spectrum of ISO 8529-1 [20].



Source: Author.

It is possible to observe that the total spectra at the distances 100 cm and 150 cm are considerably degraded and thermalized, where it is possible to observe peaks in the thermal neutron range between energies of 10^{-8} MeV and 10^{-5} MeV; this is due to the interaction of neutrons with the laboratory structure.

To characterize the neutron field, the fluence rate was determined at different positions in the LCN, through the simulation of the LC. It was positioned at the same height as the source with and without the shadow cone (between the source and the LC), in order to obtain the spectra and values of the total and direct fluence rates at the source - detector distances of 100 cm and 150 cm. Table 1 presents the results of the simulation with and without the cone and those relative to the direct beam.

Table 1: Values obtained through simulation, with and without the cone, and values related to the direct beam.

Source-detector	Fluence	
distance	rate	
(cm)	$(n/cm^2.s)$	
Beam without cone		
100	47.5 ± 2.5	
150	34.0 ± 1.7	
Beam with cone		
100	29.9 ± 1.5	
150	24.9 ± 1.2	
Direct beam		
100	17.6 ± 1.0	
150	9.3 ± 0.5	

The standard ISO 8529-2 [11] recommends that the influence of scattered radiation should not exceed 40% at the calibration point. This means that the ratio of the total fluence rate (without cone) to the scattered fluence rate (with cone) should not exceed the value recommended by the standard. The scattering fraction values at distances of 100 cm and 150 were 63% and were 88% respectively. Table 2 shows the neutron fluence rates as a function of distance for LCN and LNMRI laboratories.

Table 2: Neutron fluence rates $[cm^{-2}. s^{-1}]$ at the distances of 100 cm and 150 cm from the 241 AmBe (37 GBq) source for the LC and LNMRI laboratories [21].

Source-detector distance (cm)	LCN	LNMRI
100	17.6 ± 1.0	16.5±0.8
150	9.3 ± 0.5	8.2 ± 0.2

The percentage difference in the neutron fluence rate of the LCN at a source-detector distance of 100 cm was 6.5%, and for a distance of 150 cm it was 11.8% in relation to the LNMRI data [21].

4. CONCLUSION

In this LCN study, neutron scattering and fluence rates were determined for source-detector distances of 100 cm and 150 using Monte Carlo simulation (MCNPX) of a long counter and using the Shadow-Cone Method (SCM). Observing the spectra obtained, a decrease in the amount of fast neutrons and an increase in thermal neutrons were noticed as the source-detector distance increases.

From the results of the Long Counter (LC) simulations it is possible to observe that the direct spectra at both source-detector distances from the LCN present similarity in shape with the reference spectrum. The results obtained for LCN are similar to the LNMRI neutron calibration laboratory.

The scattering fraction values were determined using simulation at distances of 100 cm and 150 cm. The results indicate that the increase in scattering fraction is attributed to the proximity of the wall. Therefore, to ensure that the detector readings fall within the recommended range of standard ISO 8529-2 [11], it is advisable to conduct calibrations at distances below 100 cm.

ACKNOWLEDGMENTS

The authors acknowledge the Brazilian agencies CNPq (Grant No. 305142/2021-6), FAPESP (Grant No. 2018/05982-0) and CNEN (Grant No. 01342.002316/2019-46) for partial financial support.

REFERENCES

- [1] SCHUHMACHER, H. Neutron calibration facility. Radiat. Prot. Dosim. v.110, p.33-42, 2004.
- [2] LE, T.N.; TRAN, H.N.; DUONG, T.D.; NGUYEN, Q.N.; HO, T.Q.; TRINH, G.V.; NGUYEN, K.T. Evaluation of gamma contribution in a neutron calibration field of ²⁴¹AmBe source. **Appl. Radiat. Isot.** v. 133, p. 117-120, 2018.
- [3] LEE, S.K.; KIM, S.I.; LEE, J.; CHANG, I.; KIM, J.L.; KIM, H.; KIM, M.C.; KIM, B.H. Evaluation of neutron scattering correction using the Semi-Empirical Method and the Shadow-Cone Method for the neutron field of the Korea Atomic Energy Research Institute. **Radiat. Prot. Dosim.** v. 180 (1-4), p. 46-50, 2018.
- [4] PORRAS, I.; PRAENA, J.; ARIAS DE SAAVEDRA, F.; PEDROSA-RIVERA, M.; TORRES SÁNCHEZ, P.; SABARIEGO, M.P.; EXPÓSITO-HERNÁNDEZ, J.; LLAMAS-ELVIRA, J.M.; RAMÍREZ-NAVARRO, A.; RODRÍGUEZ-FERNÁNDEZ, A.; OSORIO-CEBALLOS, J.M.; RUIZ, C.; RUIZ-MAGAÑA, M.J. BNCT research activities at the Granada group and the project NeMeSis: Neutrons for medicine and sciences, towards an accelerator-based facility for new BNCT therapies, medical isotope production and other scientific neutron applications. **Appl. Radiat. Isot.** v. 165, p. 109247, 2020.
- [5] ALVARENGA, T. S.; FEDERICO, C. A.; CALDAS, L. V. E. Determination of radiation spread in the IPEN Neutron Calibration Laboratory. **Braz. J. Radiat**. **Sci.** v. 5, p. 1-10, 2017.
- [6] HWAN, B.K.; LYUL, J.K.; YOUNG, S.C., CHO, G. Scattered neutron calibration fields of KAERI. J. Nucl. Sci. Technol. v. 37, p. 781-784, 2014.

- [7] VEGA-CARRILLO, H. R.; MANZANARES-ACUÑA, E.; IÑIGUEZ, M. P.; GALLEGO, E.; LORENTE, A. Study of room-return neutrons. **Radiat. Meas.** v. 42 (3), p. 413–418, 2007.
- [8] POLA, A.; BEDOGNI, R.; DOMINGO, C.; BORTOT, D.; GOMEZ-ROS, M. J.; INTROINI, M. V.; MARTINEZ-ROVIRAC, I.; ROMERO-EXPOSITO, M.; COSTA, M. Neutron spectrometry of a lightly encapsulated ²⁴¹Americium—\Beryllium neutron source using two different Bonner Sphere Spectrometers. Nucl. Instrum. Meth. Phys. Res. Sect. A. v. 927, p. 371-374, 2019.
- [9] GRESSIER, V. Review of neutron calibration facilities and monitoring techniques: new needs for emerging fields. **Radiat. Prot. Dosim.** v. 161(1-4), p. 27-36, 2014.
- [10] DAWN, S.; BAKSHI, A. K.; SATHIAN, D.; SELVAM, T. P. Estimation of neutron scatter correction for calibration of personnel dosimeter and dose ratemeter against ²⁴¹Am-Be source Monte Carlo simulation and measurements. **Radiat. Prot. Dosim.** v. 175(2), p. 149-162, 2016.
- [11] ISO, INTERNATIONAL ORGANIZATION FOR STANDARDIZATION. Reference Neutron Radiations. Characteristics and Methods of Production. Technical Report ISO 8529- 2, Geneva, 2000.
- [12] THIEM, N. L.; HOAI-NAM, T.; KHAI, T. N.; GIAP, V. T. Neutron calibration field of a bare ²⁵²Cf source in Vietnam. **Nucl. Eng.Techn**. v. 49, p. 227-284, 2017.
- [13] ANTANACKOVIC, T.; YONKEU, A.; DUBEAU, J.; WITHARANA, H, S.; PRIEST, N. Characterization of neutron fields from bare and heavy water moderated ²⁵²Cf spontaneous fission source using Bonner sphere spectrometer. **Appl. Radiat. Isot**. v. 99, p. 122-132, 2015.
- [14] THOMAS, D. J.; NOLTE, R.; GRESSIER, V. What is neutron metrology and why is it needed? **Metrologia**. v. 48, p. 225–238, 2011.
- [15] SLAUGHTER, D. An operating Manual for the de Pangher Precision Long Couter, Lawrence Livermore Laboratory, University of California, 1978.

- [16] FERNANDES, S.S.; ASTUTO, A.; PATRÃO, K.C.S.; FONSECA, E.S.; PEREIRA,W.W.; LOPES, R.T. Long counter and its application for the calibration of the neutron irradiators. **Radiat. Prot. Dosim.** v. 180 (1-4), p. 37–41, 2018.
- [17] TAGZIRIA, H.; THOMAS, D. J. Calibration and Monte Carlo modelling of neutron long counters. **Nucl. Instrum. Meth. Phys. Res. Sect. A**. v. 452, p. 470–483, 2000.
- [18] McCONN JR, R. J.; GESH, C. J.; PAGH, R. T.; RUCKER, R.A.; WILLIAMS III, R. G. Radiation portal monitor project. Compendium of material composition data for radiation transport modeling. PNNL-15870, Rev. 1 Pacific Northwest National Laboratory, 2011.
- [19] KHABAZ, R.; HAKIMABAD, M, H. Determination of ²⁴¹Am-Be spectra using Bonner Sphere Spectrometer by applying shadow cone technique in calibration. **J. Appl. Sci**, v. 11, p. 2849-2854, 2011.
- [20] ISO INTERNATIONAL ORGANIZATION FOR STANDARDIZATION. Reference Neutron Radiations - Part 1: Characteristics and Methods of Production. Technical Report 8529-1:2001 (E), Geneva, 2001.
- [21] ROBERTS, N. J. International key comparison of measurements of neutron source emission rate (1999–2005): CCRI(III)-K9.AmBe. **Metrologia**, v. 48(S 251 Tech. Suppl.), p. 06018, 2011.

This article is licensed under a Creative Commons Attribution 4.0 International License, which permits use, sharing, adaptation, distribution and reproduction in any medium or format, as long as you give appropriate credit to the original author(s) and the source, provide a link to the Creative Commons license, and indicate if changes were made. The images or other third-party material in this article are included in the article's Creative Commons license, unless indicated otherwise in a credit line to the material.

To view a copy of this license, visit http://creativecommons.org/licenses/by/4.0